

# American Nuclear Society

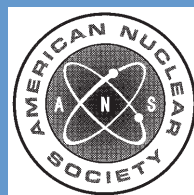
**WITHDRAWN**

April 14, 2013  
ANSI/ANS-2.10-2003

criteria for the handling and initial evaluation of records from nuclear power plant seismic instrumentation

## an American National Standard

No longer being maintained as an American National Standard. This standard may contain outdated material or may have been superseded by another standard. Please contact the ANS Standards Administrator for details.



published by the  
American Nuclear Society  
555 North Kensington Avenue  
La Grange Park, Illinois 60526 USA

ANSI/ANS-2.10-2003

**American National Standard  
Criteria for the Handling and Initial  
Evaluation of Records from Nuclear  
Power Plant Seismic Instrumentation**

Secretariat  
**American Nuclear Society**

Prepared by the  
**American Nuclear Society  
Standards Committee  
Working Group ANS-2.10**

Published by the  
**American Nuclear Society  
555 North Kensington Avenue  
La Grange Park, Illinois 60526 USA**

Approved April 14, 2003  
by the  
**American National Standards Institute, Inc.**

## **American National Standard**

Designation of this document as an American National Standard attests that the principles of openness and due process have been followed in the approval procedure and that a consensus of those directly and materially affected by the standard has been achieved.

This standard was developed under procedures of the Standards Committee of the American Nuclear Society; these procedures are accredited by the American National Standards Institute, Inc., as meeting the criteria for American National Standards. The consensus committee that approved the standard was balanced to ensure that competent, concerned, and varied interests have had an opportunity to participate.

An American National Standard is intended to aid industry, consumers, governmental agencies, and general interest groups. Its use is entirely voluntary. The existence of an American National Standard, in and of itself, does not preclude anyone from manufacturing, marketing, purchasing, or using products, processes, or procedures not conforming to the standard.

By publication of this standard, the American Nuclear Society does not insure anyone utilizing the standard against liability allegedly arising from or after its use. The content of this standard reflects acceptable practice at the time of its approval and publication. Changes, if any, occurring through developments in the state of the art, may be considered at the time that the standard is subjected to periodic review. It may be reaffirmed, revised, or withdrawn at any time in accordance with established procedures. Users of this standard are cautioned to determine the validity of copies in their possession and to establish that they are of the latest issue.

The American Nuclear Society accepts no responsibility for interpretations of this standard made by any individual or by any ad hoc group of individuals. Requests for interpretation should be sent to the Standards Department at Society Headquarters. Action will be taken to provide appropriate response in accordance with established procedures that ensure consensus on the interpretation.

Comments on this standard are encouraged and should be sent to Society Headquarters.

Published by

**American Nuclear Society  
555 North Kensington Avenue  
La Grange Park, Illinois 60526 USA**

Copyright © 2003 by American Nuclear Society. All rights reserved.

Any part of this standard may be quoted. Credit lines should read "Extracted from American National Standard ANSI/ANS-2.10-2003 with permission of the publisher, the American Nuclear Society." Reproduction prohibited under copyright convention unless written permission is granted by the American Nuclear Society.

Printed in the United States of America

## Foreword

(This Foreword is not a part of American National Standard “Criteria for the Handling and Initial Evaluation of Records from Nuclear Power Plant Seismic Instrumentation,” ANSI/ANS-2.10-2003.)

The purpose of this standard<sup>1)</sup> is to specify criteria for light-water-cooled nuclear power plants for the treatment of records obtained from seismic instrumentation specified in the American National Standard “Earthquake Instrumentation Criteria for Nuclear Power Plants,” ANSI/ANS-2.2-2002.

This standard sets forth criteria on how the records obtained from such instrumentation should be retrieved, evaluated, and stored after an earthquake. This standard provides criteria and guidelines for a timely determination for whether the operating-basis earthquake ground motion (OBE) was exceeded.

Criteria for the plant owner or its agents are provided in connection with the following activities:

- (1) retrieval of recorded data from seismic instrumentation in the event that an earthquake occurs with sufficient ground-shaking motion to activate the seismic instrumentation;
- (2) correction of the recorded data to minimize erroneous signals;
- (3) initial evaluation of the recorded data to estimate the degree of earthquake excitation to be compared with earthquake excitation values used for the plant design-basis calculations and established absolute values that are defined in this standard;
- (4) storage and maintenance of recorded data and calculations.

Currently, this standard applies to land-based, light-water-cooled nuclear power plants. This standard will be reviewed for its applicability to nuclear power plants that are based on different technologies from current light-water-cooled nuclear power plants when these technologies become available. The writing group will also review, from time to time, the standard’s applicability to new light-water-cooled power plant installations to verify that the purposes of this standard are preserved.

This standard does not address operator actions necessary to accomplish plant shutdown due to seismic activity, restart of the plant after shutdown due to seismic activity, or other supplemental power plant operator actions.

When an earthquake occurs, seismic data are recorded by the seismic instrumentation. Evaluations based on the recorded data are used to make an early determination of whether earthquake ground surface motion exceeded OBE levels of severity or the minimum damage threshold. Using this information, the guidelines in this standard can be used to determine whether the OBE was exceeded. In the case that the OBE has been exceeded, the licensee makes the decision whether to shut down the nuclear power plant. In some cases shutdown is part of the licensing basis (e.g., condition of license, Final Safety Analysis Report commitment, or Technical Specification requirement). If on the basis of

---

<sup>1)</sup>The current standard, ANSI/ANS-2.10-2003, is herein referred to as “this standard.”

this initial evaluation, and a plant walkdown (see ANSI/ANS 2.23-2002, “Nuclear Plant Response to an Earthquake”), it is concluded that the OBE seismic design has not been exceeded, it is presumed that the plant will not be shut down or be obligated to remain shut down. However, additional studies will often be undertaken to evaluate possible long-term effects of the seismic activity and to document the recorded response and the calculated results. These studies may be minimal depending on the initial evaluation and may take place while the facility is in operation.

This standard was prepared by ANS-2.10 Working Group of Subcommittee ANS-21, Nuclear Facility Design Criteria and Operations, of the American Nuclear Society Nuclear Facilities Standards Committee (NFSC).

The standard was initially approved in 1979. It was withdrawn as an American National Standard in April 1990. This newly revised standard incorporates changes in technology that have occurred since the 1979 standard was developed.

Two major changes were incorporated since the 1979 standard:

(1) this standard covers initial free-field surface record evaluation only. Evaluation of the records taken from other instruments is discussed in ANSI/ANS 2.23-2002;

(2) free-field surface motion is compared in two ways. The recorded motion response spectrum is compared with the OBE design response spectrum (or equivalent) and spectral acceleration and velocity limits. In addition, the cumulative absolute velocity of the motion is calculated and compared against a limiting damage value.

The ANS-2.10 Working Group had the following membership at the time of approval of the standard:

D. K. Ostrom, Chair, *Consultant*  
C. Angstadt, *Cleveland Electric Illuminating Company*  
M. L. Crumb, *TERRA Technology Corporation*  
J. Diel, *Agbabian and Associates*  
P. D. Engdahl, *Engdahl Enterprises*  
R. P. Kassawara, *Electric Power Research Institute*  
R. M. Kenneally, *U.S. Nuclear Regulatory Commission*  
J. McCumber, *Yankee Atomic Electric Company*  
T. O'Hara, *Yankee Atomic Electric Company*  
R. Srinivasan, *Structural Integrity Associates*  
J. Stevenson, *Stevenson and Associates*  
A. Y. C. Wong, *Stone and Webster Engineering Corporation*

<b>Contents</b>	<b>Section</b>	<b>Page</b>
	<b>1</b> Scope .....	1
	<b>2</b> Purpose .....	1
	<b>3</b> Definitions .....	1
	<b>4</b> Earthquake Evaluation Levels .....	2
	4.1 Level 1 .....	2
	4.2 Level 2 .....	2
	<b>5</b> Data Retrieval and Evaluation .....	2
	5.1 Instrument Characteristics and Status .....	2
	5.2 Data Retrieval .....	4
	5.3 Transmission of Data from the Site .....	5
	5.4 Data Reduction .....	5
	5.5 Data Evaluation .....	5
	<b>6</b> References .....	6
	<b>Figure</b>	
	Figure 1 Flowchart for Level 1 evaluation .....	3