

# American Nuclear Society

## REAFFIRMED

February 4, 2022

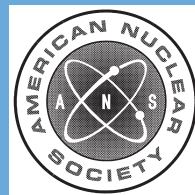
ANSI/ANS-58.14-2011 (R2022)

ANSI/ANS-58.14-2011 (R2017)

**safety and pressure integrity  
classification criteria for  
light water reactors**

## an American National Standard

This standard has been reviewed and reaffirmed with the recognition that it may reference other standards and documents that may have been superseded or withdrawn. The requirements of this document will be met by using the version of the standards and documents referenced herein. It is the responsibility of the user to review each of the references and to determine whether the use of the original references or more recent versions is appropriate for the facility. Variations from the standards and documents referenced in this standard should be evaluated and documented. This standard does not necessarily reflect recent industry initiatives for risk informed decision-making or a graded approach to quality assurance. Users should consider the use of these industry initiatives in the application of this standard.



published by the  
**American Nuclear Society**  
 555 North Kensington Avenue  
 La Grange Park, Illinois 60526 USA

**American National Standard  
Safety and Pressure Integrity  
Classification Criteria for  
Light Water Reactors**

Secretariat  
**American Nuclear Society**

Prepared by the  
**American Nuclear Society  
Standards Committee  
Working Group ANS-58.14**

Published by the  
**American Nuclear Society  
555 North Kensington Avenue  
La Grange Park, Illinois 60526 USA**

Approved April 22, 2011  
by the  
**American National Standards Institute, Inc.**

## **American National Standard**

Designation of this document as an American National Standard attests that the principles of openness and due process have been followed in the approval procedure and that a consensus of those directly and materially affected by the standard has been achieved.

This standard was developed under procedures of the Standards Committee of the American Nuclear Society; these procedures are accredited by the American National Standards Institute, Inc., as meeting the criteria for American National Standards. The consensus committee that approved the standard was balanced to ensure that competent, concerned, and varied interests have had an opportunity to participate.

An American National Standard is intended to aid industry, consumers, governmental agencies, and general interest groups. Its use is entirely voluntary. The existence of an American National Standard, in and of itself, does not preclude anyone from manufacturing, marketing, purchasing, or using products, processes, or procedures not conforming to the standard.

By publication of this standard, the American Nuclear Society does not insure anyone utilizing the standard against liability allegedly arising from or after its use. The content of this standard reflects acceptable practice at the time of its approval and publication. Changes, if any, occurring through developments in the state of the art, may be considered at the time that the standard is subjected to periodic review. It may be reaffirmed, revised, or withdrawn at any time in accordance with established procedures. Users of this standard are cautioned to determine the validity of copies in their possession and to establish that they are of the latest issue.

The American Nuclear Society accepts no responsibility for interpretations of this standard made by any individual or by any ad hoc group of individuals. Requests for interpretation should be sent to the Standards Department at Society Headquarters. Action will be taken to provide appropriate response in accordance with established procedures that ensure consensus on the interpretation.

Comments on this standard are encouraged and should be sent to Society Headquarters.

Published by

**American Nuclear Society  
555 North Kensington Avenue  
La Grange Park, Illinois 60526 USA**

Copyright © 2011 by American Nuclear Society. All rights reserved.

Any part of this standard may be quoted. Credit lines should read "Extracted from American National Standard ANSI/ANS-58.14-2011 with permission of the publisher, the American Nuclear Society." Reproduction prohibited under copyright convention unless written permission is granted by the American Nuclear Society.

Printed in the United States of America

## **Inquiry Requests**

The American Nuclear Society (ANS) Standards Committee will provide responses to inquiries about requirements, recommendations, and/or permissive statements (i.e., “shall,” “should,” and “may,” respectively) in American National Standards that are developed and approved by ANS. Responses to inquiries will be provided according to the Policy Manual for the ANS Standards Committee. Nonrelevant inquiries or those concerning unrelated subjects will be returned with appropriate explanation. ANS does not develop case interpretations of requirements in a standard that are applicable to only a specific design, operation, facility, or other unique situation and therefore is not intended for generic application.

Responses to inquiries on standards are published in ANS’s magazine, *Nuclear News*, and are available publicly on the ANS Web site or by contacting the ANS standards administrator.

## **Inquiry Format**

Inquiry requests must include the following:

- (1) the name, company name if applicable, mailing address, and telephone number of the inquirer;
- (2) reference to the applicable standard edition, section, paragraph, figure, and/or table;
- (3) the purposes of the inquiry;
- (4) the inquiry stated in a clear, concise manner;
- (5) a proposed reply, if the inquirer is in a position to offer one.

Inquiries should be addressed to

American Nuclear Society  
ATTN: Standards Administrator  
555 N. Kensington Avenue  
La Grange Park, IL 60526

or [standards@ans.org](mailto:standards@ans.org)

## Foreword

(This Foreword is not a part of American National Standard “Safety and Pressure Integrity Classification Criteria for Light Water Reactors,” ANSI/ANS-58.14-2011.)

This standard revises and supersedes the safety and pressure integrity classification criteria provided in ANSI/ANS-51.1-1983 (R1988) (withdrawn), “Nuclear Safety Criteria for the Design of Stationary Pressurized Water Reactor Plants,” and ANSI/ANS-52.1-1983 (R1988) (withdrawn), “Nuclear Safety Criteria for the Design of Stationary Boiling Water Reactor Plants.”<sup>1)</sup> The criteria in this standard are primarily objective; are applicable to all nuclear power plant functions, structures, systems, components, and parts (including consumables); and are applicable to any light water reactor (LWR) nuclear power plant design. The criteria in ANSI/ANS-51.1-1983 (R1988) (withdrawn) and ANSI/ANS-52.1-1983 (R1988) (withdrawn) are primarily subjective, apply primarily to systems, and apply only to the new designs of pressurized water reactors and boiling water reactors (BWRs) available in the United States in 1983.

This standard uses separate sets of terms for safety classification criteria<sup>2)</sup> and pressure integrity classification criteria.<sup>3)</sup> ANSI/ANS-51.1-1983 (R1988) (withdrawn) and ANSI/ANS-52.1-1983 (R1988) (withdrawn) address both safety and pressure integrity classification criteria using a single set of terms (Safety Classes 1, 2, and 3, and Non-Nuclear Safety). The applicability of these two sets of criteria is not identical. The single set of terms used in ANSI/ANS-51.1-1983 (R1998) (withdrawn) and ANSI/ANS-52.1-1983 (R1988) (withdrawn) creates inconsistencies and a potential for misinterpretations. These limitations are avoided in this standard.

The safety classification criteria in this standard are based on NEDC-31509, “Safety Classification Methodology and Criteria for Structures, Systems, Components and Parts in BWR Nuclear Power Plants,” developed by the Parts Safety Classification Committee of the BWR Owners Group and GE Nuclear Energy, and on EPRI NP-6895, “Guidelines for the Safety Classification of Systems, Components and Parts Used in Nuclear Power Plant Applications (NCIG-17),” developed by the Nuclear Construction Issues Group, a utility group sponsored by the Electric Power Research Institute.

The application of many requirements to nuclear power plant structures, systems, components, and parts is based upon their safety classification. The safety classification of an item is typically used to determine which design, procurement, manufacturing, construction, and operating requirements or controls apply.

The term “safety-related” is used to identify items that, because of their functional safety importance, must meet stringent design requirements such as Seismic Category I criteria; IEEE Class 1E criteria for electrical items; *ASME Boiler and Pressure Vessel Code*, Sec. III, criteria for pressure integrity items; and environmental qualification requirements of *Code of Federal Regulations*, Title 10, “Energy,” Part 50, “Domestic Licensing of Production and Utilization Facilities,” Sec. 49, “Environmental Qualification of Electric Equipment Important to Safety for Nuclear Power Plants.”

The safety classification of an item might be also used to help establish the procurement requirements for the item. Typically, there are three types of procurement classifications: safety-related, commercial grade, and non-safety-related.

---

<sup>1)</sup> The remaining portions of ANSI/ANS-51.1-1983 (R1988) (withdrawn) and ANSI/ANS-52.1-1983 (R1988) (withdrawn) are not superseded by this standard.

<sup>2)</sup> Safety-related (Q), non-safety-related with augmented quality assurance [or augmented (A)], and non-safety-related (N).

<sup>3)</sup> Classes 1, 2, 3, 4, and 5.

A safety-related procurement refers to an item that is purchased subject to the provisions of *Code of Federal Regulations*, Title 10, “Energy,” Part 21, “Reporting of Defects and Noncompliance” (10 CFR 21), and is intended for use in applications that are functionally safety-related. Commercial-grade procurement refers to an item that is purchased without the provisions of 10 CFR 21 but is intended to be dedicated after receipt for use in applications that are functionally safety-related. Once a commercial-grade item is dedicated, it becomes a safety-related item. Non-safety-related procurement refers to an item that is purchased without the provisions of 10 CFR 21 and is intended for use in applications that are functionally non-safety-related.

During construction, safety-related items are subject to specific material selection, design, fabrication, examination, testing, inspection, certification, installation, and quality assurance requirements.

Operationally, safety-related items typically are subject to specified requirements for in-service inspection, in-service testing, maintenance, surveillance, and quality assurance.

The classification “non-safety-related with augmented requirements” is applied to certain non-safety-related items during procurement, construction, and operations when the item is not safety-related but is relied upon during a special event or where licensing requirements exist.

The focus of safety classification in this standard is on the accomplishment of safety-related functions that may be considered to provide design-basis-event prevention, mitigation, or both, without emphasizing one over the other. Multiple redundant levels of defense provide a balance of defense-in-depth strategies such that no single element (e.g., accident prevention) or barrier (e.g., containment) is emphasized to the exclusion of others. An adequate balance of prevention and mitigation as well as consideration of defense in depth is provided in the classification process through the consideration of the three basic safety-related functions identified in *Code of Federal Regulations*, Title 10, “Energy,” Part 50, “Domestic Licensing of Production and Utilization Facilities,” Sec. 2, “Definitions.”

The pressure integrity classification criteria provided in Sec. 5 are similar to those of ANSI/ANS-51.1-1983 (R1988) (withdrawn), ANSI/ANS-52.1-1983 (R1988) (withdrawn), and Regulatory Guide 1.26, “Quality Group Classifications and Standards for Water-, Steam-, and Radioactive-Waste-Containing Components of Nuclear Power Plants,” but have been revised to be applicable to any LWR design (particularly an advanced passive design).

The basic design requirements for items assigned to each safety classification are summarized in Sec. 6.

This standard has been written for prospective use, but the criteria are based on current practices and requirements applicable to licensed LWR designs.

This standard might reference documents and other standards that have been superseded or withdrawn at the time the standard is applied. A statement has been included in the references section (Sec. 7) that provides guidance on the use of references.

This standard does not incorporate the concepts of generating risk-informed insights, performance-based requirements, or a graded approach to quality assurance. The user is advised that one or more of these techniques could enhance the application of this standard.

Working Group ANS-58.14 of the Standards Committee of the American Nuclear Society had the following membership at the time it developed this standard:

M. A. Linn (Chair), *Oak Ridge National Laboratory*

D. P. Blanchard, *Applied Reliability Engineering, Inc.*  
S. A. Highley, *AREVA NP, Inc.*  
R. A. Hill, *ERIN Engineering and Research, Inc.*  
G. B. Locklear, *Engineering & Management Specialists, Inc.*  
P. A. Sicard, *Entergy*  
R. B. Williston, *Individual*

Subcommittee ANS-22, Systems Design Criteria, had the following membership at the time of its approval of this standard:

D. G. Newton, (Chair), *AREVA NP, Inc.*  
R. M. Ruby (Vice Chair), *Constellation Energy*  
N. W. Brown, *Individual*  
R. A. Hill, *ERIN Engineering and Research, Inc.*  
L. E. Kreider, *Engineering Planning & Management, Inc.*  
M. A. Linn, *Oak Ridge National Laboratory*  
D. W. Murphy, *Bechtel Corporation*

The Nuclear Facilities Standards Committee (NFSC) had the following membership at the time of its approval of this standard:

C. A. Mazzola (Chair), *Shaw Environmental & Infrastructure, Inc.*  
R. M. Ruby (Vice Chair), *Constellation Energy Company*  
J. A. August, *CORE, Inc.*  
W. H. Bell, *South Carolina Electric & Gas Company*  
J. R. Brault, *Shaw MOX Project*  
C. K. Brown, *Southern Nuclear Operating Company*  
K. R. Bryson, *Shaw Environmental, Inc.*  
C. E. Carpenter, *U.S. Nuclear Regulatory Commission*  
(Alt. A. Hull, *U.S. Nuclear Regulatory Commission*)  
D. R. Eggett, *Automated Engineering Services Corporation*  
R. W. Englehart, *Individual*  
P. K. Guha, *U.S. Department of Energy*  
(Atl. J. O'Brien, *U.S. Department of Energy*)  
P. S. Hastings, *Duke Energy Company (NuStart Liaison)*  
R. A. Hill, *ERIN Engineering and Research, Inc.*  
N. P. Kadambi, *Individual*  
E. M. Lloyd, *Exitech Corporation*  
S. A. Lott, *Los Alamos National Laboratory*  
R. H. McFetridge, *Westinghouse Electric Company, LLC*  
(Alt. L. R. Grobmyer, *Westinghouse Electric Company, LLC*)  
T. K. Meneely, *Westinghouse Electric Company, LLC*  
(Alt. R. C. Surman, *Westinghouse Electric Company, LLC*)  
C. H. Moseley, *ASME NQA Liaison*  
D. G. Newton, *AREVA NP*  
W. N. Prillaman, *AREVA NP*  
W. B. Reuland, *Individual*  
J. C. Saldarini, *Bechtel SAIC Company, LLC*  
(Alt. A. T. Vieira, *Bechtel Power Corporation*)  
D. J. Spellman, *Oak Ridge National Laboratory (NFSC Liaison to IEEE NPEC)*  
S. L. Stamm, *Shaw Nuclear Services*  
J. D. Stevenson, *Individual*  
J. A. Wehrenberg, *Southern Company Services*  
M. J. Wright, *Entergy Operations, Inc.*

NFSC Liaison:

G. Hutcherson, *Institute of Nuclear Power Operations*  
J. H. Riley, *Nuclear Energy Institute*  
(Alt. J. C. Butler, *Nuclear Energy Institute*)

NFSC Observers:

R. H. Bryan, *Tennessee Valley Authority*  
E. P. Loewen, *General Electric*  
J. E. Love, *Bechtel Power Corporation*  
C. D. Thomas, *Individual*  
R. E. Scott, *Individual*

<b>Contents</b>	<b>Section</b>	<b>Page</b>
<b>1</b>	<b>Introduction</b> .....	<b>1</b>
1.1	Scope .....	1
1.2	Purpose .....	1
1.3	Applicability .....	1
<b>2</b>	<b>Acronyms and definitions</b> .....	<b>2</b>
2.1	List of acronyms .....	2
2.2	Definitions .....	2
<b>3</b>	<b>General requirements</b> .....	<b>6</b>
3.1	Regulatory basis .....	6
3.2	Relationship of functional, procurement, and application safety classifications .....	7
3.3	Methodology .....	7
<b>4</b>	<b>Safety classification criteria</b> .....	<b>9</b>
4.1	General criteria .....	9
4.2	Determination of design basis events .....	11
4.3	Determination of safety-related functions .....	11
4.4	Determination of safety-related systems and structures .....	12
4.5	Determination of safety-related components and parts .....	13
4.6	Items not classified safety-related .....	21
<b>5</b>	<b>Pressure integrity classification criteria</b> .....	<b>23</b>
5.1	Functional criteria .....	23
5.2	Interface criteria .....	24
<b>6</b>	<b>Basic design requirements</b> .....	<b>25</b>
6.1	Pressure-retaining items .....	26
6.2	Electrical items .....	26
6.3	Seismic classification .....	26
6.4	Environmental qualification .....	26
6.5	Quality assurance .....	26
<b>7</b>	<b>References</b> .....	<b>26</b>
<b>Appendices</b>		
Appendix A	Relationship of the Terms “Safety-Related” and “Safety” ...	30
Appendix B	Classification Categories .....	32
Appendix C	Safety Classification Examples .....	36
Appendix D	Typical Examples of Interfaces Between Pressure Integrity Classes .....	60
<b>Tables</b>		
Table 1	Basic design requirements .....	25
Table A.1	Approximate relationship of various safety classification terms .....	30
<b>Figures</b>		
Figure 1	Methodology for safety classification .....	7
Figure 2	Safety-related/non-safety-related fluid system boundary criteria .....	15



Figure 3	Boundary criteria for fluid system lines penetrating primary containment .....	16
Figure 4	Boundary criteria for instrument lines connected to the RCPB and penetrating primary containment .....	19
Figure 5	Boundary criteria for instrument lines not penetrating primary containment (Note 1) .....	20
Figure C.1	Auxiliary feedwater system (examples 2 and 3) .....	39
Figure C.2	Battery room heat removal system (examples 4 and 5) ....	43
Figure C.3	Auxiliary power system synchronizing circuit (example 8) ..	47
Figure C.4	Hydraulic snubber seals (example 21) .....	58
Figure D.1	Typical fluid-system pressure integrity class interfaces ...	61