

ERRATUM ISSUED

Find erratum inside front cover. If erratum is missing, contact the ANS Standards Department at Standards@ans.org or 708-579-8269 for replacement copy.

American Nuclear Society

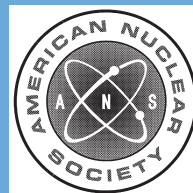
nuclear criticality safety in operations with fissionable materials outside reactors

an American National Standard

REAFFIRMED

June 5, 2023
November 29, 2018
ANSI/ANS-8.1-2014 (R2023)

This standard has been reviewed and reaffirmed with the recognition that it may reference other standards and documents that may have been superseded or withdrawn. The requirements of this document will be met by using the version of the standards and documents referenced herein. It is the responsibility of the user to review each of the references and to determine whether the use of the original references or more recent versions is appropriate for the facility. Variations from the standards and documents referenced in this standard should be evaluated and documented. This standard does not necessarily reflect recent industry initiatives for risk informed decision-making or a graded approach to quality assurance. Users should consider the use of these industry initiatives in the application of this standard.



published by the
American Nuclear Society
555 North Kensington Avenue
La Grange Park, Illinois 60526 USA

Erratum

ANSI/ANS-8.1-2014 (R2023)

Nuclear Criticality Safety in Operations with Fissionable Material Outside Reactors

A typographical error was identified in footnote 5, on page 5. The reference to the sphere size in the last sentence should be “1/8-inch” instead of “108-inch.” The corrected footnote is below.

⁵⁾ In the homogeneous mixtures to which calculations of these subcritical limits were normalized, the average particle size of dry UO_3 was $60\ \mu\text{m}$ [V. I. Neeley and H. E. Handler, “Measurement of Multiplication Constant for Slightly Enriched Homogeneous UO_3 -Water Mixtures and Minimum Enrichment for Criticality,” HW-70310, Hanford Atomic Products Operations (Aug. 1961)]. It seems likely that the average particle size of the dihydrate of $\text{UO}_2(\text{NO}_3)_2$ was $\sim 100\ \mu\text{m}$ [V. I. Neeley, J. A. Berberet, and R. H. Masterson, “ k_∞ of Three Weight Per Cent ^{235}U Enriched UO_3 and $\text{UO}_2(\text{NO}_3)_2$ Hydrogenous Systems,” HW-66882, Hanford Atomic Products Operations (Sep. 1961)]. Various H/U ratios in the nitrate mixtures were achieved with 1/8-in. spheres of polyethylene [S. R. Bierman and G. M. Hess, “Minimum Critical ^{235}U Enrichment of Homogeneous Uranyl Nitrate,” ORNLDC-5, Oak Ridge Criticality Data Center (June 1968)].

**American National Standard
Nuclear Criticality Safety in Operations with
Fissionable Materials Outside Reactors**

Secretariat
American Nuclear Society

Prepared by the
**American Nuclear Society
Standards Committee
Working Group ANS-8.1**

Published by the
**American Nuclear Society
555 North Kensington Avenue
La Grange Park, Illinois 60526 USA**

Approved April 15, 2014
by the
American National Standards Institute, Inc.

American National Standard

Designation of this document as an American National Standard attests that the principles of openness and due process have been followed in the approval procedure and that a consensus of those directly and materially affected by the standard has been achieved.

This standard was developed under the procedures of the Standards Committee of the American Nuclear Society; these procedures are accredited by the American National Standards Institute, Inc., as meeting the criteria for American National Standards. The consensus committee that approved the standard was balanced to ensure that competent, concerned, and varied interests have had an opportunity to participate.

An American National Standard is intended to aid industry, consumers, governmental agencies, and general interest groups. Its use is entirely voluntary. The existence of an American National Standard, in and of itself, does not preclude anyone from manufacturing, marketing, purchasing, or using products, processes, or procedures not conforming to the standard.

By publication of this standard, the American Nuclear Society does not insure anyone utilizing the standard against liability allegedly arising from or after its use. The content of this standard reflects acceptable practice at the time of its approval and publication. Changes, if any, occurring through developments in the state of the art, may be considered at the time that the standard is subjected to periodic review. It may be reaffirmed, revised, or withdrawn at any time in accordance with established procedures. Users of this standard are cautioned to determine the validity of copies in their possession and to establish that they are of the latest issue.

The American Nuclear Society accepts no responsibility for interpretations of this standard made by any individual or by any ad hoc group of individuals. Responses to inquiries about requirements, recommendations, and/or permissive statements (i.e., “shall,” “should,” and “may,” respectively) should be sent to the Standards Department at Society Headquarters. Action will be taken to provide appropriate response in accordance with established procedures that ensure consensus.

Comments on this standard are encouraged and should be sent to Society Headquarters.

Published by

**American Nuclear Society
555 North Kensington Avenue
La Grange Park, Illinois 60526 USA**



This document is copyright protected.

Copyright © 2014 by American Nuclear Society. All rights reserved.

Any part of this standard may be quoted. Credit lines should read “Extracted from American National Standard ANSI/ANS-8.1-2014 with permission of the publisher, the American Nuclear Society.” Reproduction prohibited under copyright convention unless written permission is granted by the American Nuclear Society.

Printed in the United States of America.

Inquiry Requests

The American Nuclear Society (ANS) Standards Committee will provide responses to inquiries about requirements, recommendations, and/or permissive statements (i.e., “shall,” “should,” and “may,” respectively) in American National Standards that are developed and approved by ANS. Responses to inquiries will be provided according to the Policy Manual for the ANS Standards Committee. Nonrelevant inquiries or those concerning unrelated subjects will be returned with appropriate explanation. ANS does not develop case interpretations of requirements in a standard that are applicable to a specific design, operation, facility, or other unique situation only, and therefore is not intended for generic application.

Responses to inquiries on standards are published in the Society’s magazine, *Nuclear News*, and are available publicly on the ANS Web site or by contacting the ANS standards administrator.

Inquiry Format

Inquiry requests must include the following:

- (1) the name, company name if applicable, mailing address, and telephone number of the inquirer;
- (2) reference to the applicable standard edition, section, paragraph, figure, and/or table;
- (3) the purposes of the inquiry;
- (4) the inquiry stated in a clear, concise manner;
- (5) a proposed reply, if the inquirer is in a position to offer one.

Inquiries should be addressed to:

American Nuclear Society
ATTN: Standards Administrator
555 N. Kensington Avenue
La Grange Park, IL 60526

or standards@ans.org

Foreword (This Foreword is not a part of American National Standard “Nuclear Criticality Safety in Operations with Fissionable Materials Outside Reactors,” ANSI/ANS-8.1-2014.)

This standard provides guidance for the prevention of criticality accidents in the handling, storing, processing, and transportation of fissionable material. It was first approved as American Standard N6.1-1964. A substantial revision that included the specification of subcritical limits applicable to process variables was approved as American National Standard N16.1-1969 and was affirmed, with minor revisions, as American National Standard N16.1-1975/ANS-8.1. It was subsequently supplemented by American National Standard for Validation of Computational Methods for Nuclear Criticality Safety, ANSI N16.9-1975/ANS-8.11. The two standards were consolidated in 1983.

The subcritical limits given in the standard make no allowance for operating contingencies (e.g., double batching) or for inaccurate knowledge of process variables (e.g., concentrations, masses, and dimensions) and are “maximum subcritical limits” for the stated conditions. That is, under the stated conditions, the limits are close enough to critical to provide little incentive for attempting to justify slightly larger values, but concomitantly, they are confidently expected actually to be subcritical. The stated conditions (infinitely long cylinders, absence of neutron-absorbing vessel wall, plutonium solutions without free nitric acid, etc.) are unlikely to be approached in practice; hence, if a limit is reached, there will ordinarily be a larger margin of subcriticality than the minimal value used in its derivation. However, no account was taken of this unlikelihood in setting the limits. It is legitimate for the users of the standard, if they so choose, to make conservative adjustments in the limits to take advantage of the extent to which process conditions may deviate from stated conditions, e.g., to increase a cylinder diameter limit to take advantage of a finite height and of neutron absorption in steel walls.

The present revision of the standard is primarily intended to clarify the use and interpretation of the process analysis requirement, the double-contingency-principle recommendation, and their relationship in a new Appendix. In addition, the definitions for “parameter” and “process conditions” were added to assist with the understanding of the double-contingency recommendation. These and other minor changes were made that do not change the intent of the words in the previous revision. They represent clarification and amplification that should aid in uniform application of the standard.

This standard might reference documents and other standards that have been superseded or withdrawn at the time the standard is applied. A statement has been included in the references section that provides guidance on the use of references.

This standard does not incorporate the concepts of generating risk-informed insights, performance-based requirements, or a graded approach to quality assurance. The user is advised that one or more of these techniques could enhance the application of this standard.

The working group would like to gratefully acknowledge the contributions by Terry L. Hofer, who died prior to the publication of this revision.

This revision of American National Standard ANSI/ANS-8.1-2014 was prepared by Working Group ANS-8.1 of Subcommittee 8 of the Standards Committee of the American Nuclear Society. Working Group ANS-8.1 had the following membership at the time of the revision:

D. G. Bowen (Cochair), *Oak Ridge National Laboratory*
N. W. Brown (Cochair), *Nuclear Fuel Services, Inc.*

R. Beck, *USEC Incorporated*
A. Garcia, *U.S. Department of Energy*

S. Goluoglu, *University of Florida*
C. Gross, *Paschal Solutions Incorporated*
C. Haught, *B&W Y-12*
J. Hicks, *U.S. Department of Energy*
T. Hofer, *USEC Incorporated*
M. LeTellier, *C. S. Engineering, Inc.*
T. Marenchin, *U.S. Nuclear Regulatory Commission*
J. Miller, *Sigma Science, Inc.*
L. Montierth, *Idaho National Laboratory*
J. Morman, *Argonne National Laboratory*
L. Paschal, *Paschal Solutions Incorporated*
L. Paulson, *GE Hitachi, Nuclear Energy*
K. Reynolds, *B&W Y-12*
E. Saylor, *Oak Ridge National Laboratory*
F. Winstanley, *Sellafield Ltd.*
K. Woods, *Paschal Solutions Incorporated*

This standard was prepared under the guidance of ANS Subcommittee 8, which had the following membership at the time of its approval:

T. P. McLaughlin (Chair), *Individual*
B. O. Kidd (Vice Chair), *Babcock & Wilcox*
M. Crouse (Secretary), *Washington Safety Management Solutions*

F. M. Alcorn, *Individual*
J. Baker, *Los Alamos National Laboratory*
E. Elliott, *Defense Nuclear Facilities Safety Board*
D. Erickson, *Savannah River Nuclear Solutions*
A. S. Garcia, *U.S. Department of Energy*
D. A. Reed, *Individual*
T. A. Reilly, *Individual*
H. Toffer, *Individual*
C. Tripp, *U.S. Nuclear Regulatory Commission*
F. Winstanley, *Sellafield Ltd.*

The American Nuclear Society Nuclear Criticality Safety Consensus Committee had the following membership at the time of its approval:

R. D. Busch (Chair), *University of New Mexico*
L. L. Wetzel (Vice Chair), *Babcock & Wilcox Nuclear Operations Group*
L. Berg, *U.S. Department of Energy*
G. H. Bidinger, *Individual*
R. S. Eby, *American Institute of Chemical Engineers (Employed by USEC, Inc.)*
C. M. Hopper, *Individual*
R. Knief, *Institute of Nuclear Materials Management (Employed by Sandia National Laboratories)*
W. Doane, *AREVA*
T. Marenchin, *U.S. Nuclear Regulatory Commission*
T. P. McLaughlin, *Individual*
S. P. Murray, *Health Physics Society (Employed by General Electric)*
R. E. Pevey, *University of Tennessee*
R. L. Reed, *URS Professional Solutions LLC*
W. R. Shackelford, *Nuclear Fuel Services, Inc.*
R. G. Taylor, *INM Nuclear Safety Services*
R. M. Westfall, *Oak Ridge National Laboratory*
L. L. Wetzel, *Babcock & Wilcox Nuclear Operations Group*
R. E. Wilson, *U.S. Department of Energy*

Contents	Section	Page
	1 Introduction	1
	2 Scope	1
	3 Definitions.....	1
	3.1 Limitations	1
	3.2 Shall, should, and may.....	1
	3.3 Glossary of terms.....	1
	4 Nuclear criticality safety practices.....	2
	4.1 Administrative practices.....	2
	4.1.1 Responsibilities	2
	4.1.2 Process analysis	2
	4.1.3 Written procedures.....	2
	4.1.4 Materials control.....	2
	4.1.5 Operational control	3
	4.1.6 Operational reviews	3
	4.1.7 Emergency procedures.....	3
	4.2 Technical practices	3
	4.2.1 Controlled parameters.....	3
	4.2.2 Double-contingency principle	3
	4.2.3 Geometry control.....	3
	4.2.4 Neutron absorbers.....	3
	4.2.5 Moderation	3
	4.2.6 Other.....	4
	4.2.7 Subcritical limit.....	4
	4.3 Validation of a calculational method.....	4
	5 Single-parameter limits for fissile nuclides	4
	5.1 Uniform aqueous solutions	5
	5.2 Aqueous mixtures.....	5
	5.2.1 Enrichment subcritical limits.....	5
	5.3 Metallic units	5
	5.4 Oxides	7
	6 Multiparameter control.....	7
	6.1 Uranium metal–water mixture and uranium oxide–water mixture at low ²³⁵ U enrichment	7
	6.2 Aqueous uranium solutions at low ²³⁵ U enrichment	7
	6.3 Uniform aqueous solutions of Pu(NO ₃) ₄ containing ²⁴⁰ Pu	7
	6.4 Aqueous mixtures of plutonium containing ²⁴⁰ Pu.....	11
	7 References.....	11
Appendices		
	Appendix A.....	13
	Appendix B.....	15
Figures		
	Figure 1 Subcritical mass limits for uranium-water lattices	8
	Figure 2 Subcritical cylinder diameter limits for uranium-water lattices	8

Figure 3	Subcritical slab thickness limits for uranium-water lattices	9
Figure 4	Subcritical volume limits for uranium-water lattices.....	9
Figure 5	Subcritical areal density limits for uranium-water lattices	10

Tables

Table 1	Single-parameter subcritical limits for uniform aqueous solutions of fissile nuclides.....	4
Table 2	²³⁵ U enrichment subcritical limits for uranium mixed homogeneously with water.....	5
Table 3	Single-parameter subcritical limits for metal units.....	5
Table 4	Single-parameter subcritical limits for oxides containing no more than 1.5% water by weight at full density.....	6
Table 5	Single-parameter subcritical limits for oxides containing no more than 1.5% water by weight at no more than half density.....	6
Table 6	Subcritical limits for uniform aqueous solutions of low-enriched uranium.....	10
Table 7	Subcritical limits for uniform aqueous solutions of of Pu(NO ₃) ₄ containing ²⁴⁰ Pu.....	11