



Nuclear Data Sets for Reactor Design Calculations

An American National Standard



American National Standard Nuclear Data Sets for Reactor Design Calculations

Secretariat
American Nuclear Society

Prepared by the American Nuclear Society Standards Committee Working Group ANS-19.1

Published by the American Nuclear Society 555 North Kensington Avenue La Grange Park, Illinois 60526 USA

Approved March 8, 2019 by the **American National Standards Institute, Inc.**

American National Standard

Designation of this document as an American National Standard attests that the principles of openness and due process have been followed in the approval procedure and that a consensus of those directly and materially affected by the standard has been achieved.

This standard was developed under the procedures of the Standards Committee of the American Nuclear Society; these procedures are accredited by the American National Standards Institute, Inc., as meeting the criteria for American National Standards. The consensus committee that approved the standard was balanced to ensure that competent, concerned, and varied interests have had an opportunity to participate.

An American National Standard is intended to aid industry, consumers, governmental agencies, and general interest groups. Its use is entirely voluntary. The existence of an American National Standard, in and of itself, does not preclude anyone from manufacturing, marketing, purchasing, or using products, processes, or procedures not conforming to the standard.

By publication of this standard, the American Nuclear Society does not insure anyone utilizing the standard against liability allegedly arising from or after its use. The content of this standard reflects acceptable practice at the time of its approval and publication. Changes, if any, occurring through developments in the state of the art, may be considered at the time that the standard is subjected to periodic review. It may be reaffirmed, revised, or withdrawn at any time in accordance with established procedures. Users of this standard are cautioned to determine the validity of copies in their possession and to establish that they are of the latest issue.

The American Nuclear Society accepts no responsibility for interpretations of this standard made by any individual or by any ad hoc group of individuals. Inquiries about requirements, recommendations, and/or permissive statements (i.e., "shall," "should," and "may," respectively) should be sent to the Publications and Standards Department at Society Headquarters. Action will be taken to provide appropriate response in accordance with established procedures that ensure consensus.

Comments on this standard are encouraged and should be sent to Society Headquarters.

Published by

American Nuclear Society 555 North Kensington Avenue La Grange Park, Illinois 60526 USA



This document is copyright protected.

Copyright © 2019 by American Nuclear Society. All rights reserved.

Any part of this standard may be quoted. Credit lines should read "Extracted from American National Standard ANSI/ANS-19.1-2019 with permission of the publisher, the American Nuclear Society." Reproduction prohibited under copyright convention unless written permission is granted by the American Nuclear Society.

Printed in the United States of America

Inquiry Requests

The American Nuclear Society (ANS) Standards Committee will provide responses to inquiries about requirements, recommendations, and/or permissive statements (i.e., "shall," "should," and "may," respectively) in American National Standards that are developed and approved by ANS. Responses to inquiries will be provided according to the Policy Manual for the ANS Standards Committee. Nonrelevant inquiries or those concerning unrelated subjects will be returned with appropriate explanation. ANS does not develop case interpretations of requirements in a standard that are applicable to a specific design, operation, facility, or other unique situation only and therefore is not intended for generic application.

Responses to inquiries on standards are published in ANS's magazine, *Nuclear News*, and are available publicly on the ANS website or by contacting the Publications and Standards Department.

Inquiry Format

Inquiry requests shall include the following:

- (1) the name, company name if applicable, mailing address, and telephone number of the inquirer;
- (2) reference to the applicable standard edition, section, paragraph, figure, and/or table;
- (3) the purpose(s) of the inquiry;
- (4) the inquiry stated in a clear, concise manner;
- (5) a proposed reply, if the inquirer is in a position to offer one.

Inquiries should be addressed to:

American Nuclear Society Publications and Standards Department 555 N. Kensington Avenue La Grange Park, IL 60526

or standards@ans.org

American National Standard ANSI/ANS-19.1-2019

Foreword

(This foreword is not a part of American National Standard "Nuclear Data Sets for Reactor Design Calculations," ANSI/ANS-19.1-2019, but is included for informational purposes.)

The intent of this American National Standard is to provide guidance for the preparation of nuclear data sets for use in computer programs employed in the design of nuclear reactors. The nuclear data sets considered are evaluated data sets, processed continuous data sets, and processed averaged data sets. The nature of nuclear data sets considered in this standard is applicable to any reactor type. This standard is intended primarily for nuclear data to be used in reactor core calculations; however, the data may also be useful for other applications, such as shielding and dosimetry.

For evaluated data sets, guidance is provided concerning data sources, preparation of the evaluation, estimation of accuracy, verification, testing, and documentation. For processed (continuous or averaged) data sets, guidance is provided concerning data sources and data set preparation, checking, validation, and documentation.

The intent of this standard is not only to provide guidance on the creation of nuclear data sets, but also to require reactor analysts to (1) carefully consider the source, accuracy, and applicability of the nuclear data sets used in their simulations; (2) ensure that those data sets have been appropriately validated for the particular application; and (3) document the basis for confidence in decisions made regarding use of specific nuclear data sets. Compliance with the intent of this standard can be demonstrated through documentation that addresses all requirements of the standard.

Unlike ANS-19.1-2002 (R2011) (W2019), withdrawn with the approval of the 2019 version, no data sets are identified as standard data sets. However, several widely used data sets are listed in the appendix.

This standard might reference documents and other standards that have been superseded or withdrawn at the time the standard is applied. A statement has been included in the references section that provides guidance on the use of references.

This standard does not incorporate the concepts of generating risk-informed insights, performance-based requirements, or a graded approach to quality assurance. The user is advised that one or more of these techniques could enhance the application of this standard.

This standard was prepared by the ANS-19.1 Working Group of the American Nuclear Society. The following members contributed to this standard:

- R. C. Little (Chair), Los Alamos National Laboratory
- F. A. Alpan, Westinghouse Electric Company, LLC
- S. P. Baker, Transware Enterprises
- J. D. Bess, Idaho National Laboratory
- D. M. Cokinos, Brookhaven National Laboratory
- D. E. Cullen, Individual
- M. E. Dunn, Spectra Tech, Inc.
- M. A. Garland, U.S. Department of Energy
- I. C. Gauld, Oak Ridge National Laboratory
- M. W. Herman, Brookhaven National Laboratory
- A. C. Kahler, Los Alamos National Laboratory
- E. R. Knuckles, Individual
- R. D. Mosteller, Individual
- G. Radulescu, Oak Ridge National Laboratory
- B. Rouben, Individual
- M. L. Zerkle, Naval Nuclear Laboratory

The Reactor Physics Subcommittee had the following membership at the time of its approval of this standard:

- D. M. Cokinos (Chair), Brookhaven National Laboratory
- C. T. Rombough (Secretary), CTR Technical Services, Inc.
- A. C. Attard, U.S. Nuclear Regulatory Commission
- S. P. Baker, Transware Enterprises
- J. D. Bess, *Idaho National Laboratory*
- A. Campos, Framatome
- R. T. Chiang, *Individual*
- M. DeHart, Idaho National Laboratory
- D. J. Diamond, Brookhaven National Laboratory
- M. Eckenrode, Framatome
- I. C. Gauld, Oak Ridge National Laboratory
- A. Haghighat, Virginia Tech Research Center
- J. I. Katakura, Japan Atomic Energy Agency
- E. R. Knuckles, Individual
- R. C. Little, Los Alamos National Laboratory
- M. Mahgerefteh, Exelon Corporation
- E. M. Nichita, University of Ontario Institute of Technology
- G. Radulescu, Oak Ridge National Laboratory
- B. Rouben, Individual
- A. Weitzberg, Individual
- W. Wilson, Individual

The Safety and Radiological Analysis Consensus Committee had the following membership at the time of its approval of this standard:

- A. O. Smetana (Chair), Savannah River National Laboratory
- J. M. Jarvis (Vice Chair), Bechtel Corporation
- F. A. Alpan, Westinghouse Electric Company, LLC
- R. S. Amato, Individual
- D. M. Cokinos, Brookhaven National Laboratory
- D. J. Dudziak, Los Alamos National Laboratory
- C. C. Graham, Health Physics Society Representative (Employed by Ameren)
- M. K. Gupta, AECOM-Professional Solutions
- N. E. Hertel, Georgia Institute of Technology
- P. Hulse, Sellafield Ltd.
- D. E. Palmrose, U.S. Nuclear Regulatory Commission
- C. T. Rombough, CTR Technical Services, Inc.
- C. E. Sanders, University of Nevada, Las Vegas
- A. Weitzberg, Individual

Contents

Section P							
1	Intro	duction		1			
2	Scor	Scope					
	2.1		al				
	2.2 Applications						
2	Dofi	• •					
3	3.1	Definitions					
	3.1		should, and may				
	3.3		itions				
	3.4		nyms				
			•				
4	Rela	tion to c	other standards	3			
5	Eval		ata sets				
	5.1		al				
	5.2	Data s	sources				
		5.2.1	Experimental microscopic data				
		5.2.2	Fission product yields				
		5.2.3	Nuclear model codes				
	5.3	-	ration				
		5.3.1	Data representation				
		5.3.2	Data types				
		5.3.3	Resonance data				
		5.3.4	Continuum region data				
		5.3.5	Thermal region data				
		5.3.6	Thermal neutron scattering data				
	5.4		ation of accuracy				
	5.5		cation				
		5.5.1	Clerical and numerical accuracy				
		5.5.2	Internal consistency				
		5.5.3	Correlated evaluations				
		5.5.4	Completeness				
	5.6		ng				
		5.6.1	Independent review				
		5.6.2	Comparison with integral measurements				
		5.6.3	1				
	5.7		mentation				
		5.7.1	Documentation of data sources				
		5.7.2	Documentation of evaluation techniques				
		5.7.3	Documentation of integral testing				
		5.7.4	Documentation of benchmark testing	8			
6	Processed continuous data sets						
	6.1	6.1 General					
	6.2	e	8				
	6.3	Prepar	ration	8			
		6.3.1	Data representation	8			
		6.3.2	Data types	9			
		6.3.3	Accuracy	9			

	6.4	Check	xing	9			
		6.4.1	Clerical and numerical accuracy	9			
		6.4.2	Internal consistency				
		6.4.3	Consistency with evaluated data				
		6.4.4	Alternate processing codes				
	6.5	Valida	ation				
		6.5.1	Appropriate physical configurations	9			
		6.5.2	Applicable integral measurements	10			
		6.5.3					
	6.6	Docur	mentation				
		6.6.1	Documentation of data sources	10			
		6.6.2	Documentation of processing techniques	10			
		6.6.3	Documentation of integral testing				
		6.6.4	Documentation of benchmark testing				
		6.6.5	Documentation of expected range of qualified application				
_	ъ.						
7		Processed averaged data sets					
	7.1		al				
	7.2		es				
	7.3	-	ration				
		7.3.1	- T				
			Data types				
		7.3.3	\mathcal{E}				
		7.3.4	<i>C</i> ; <i>C</i>				
		7.3.5	J				
	7.4		xing				
		7.4.1	<i>y</i>				
		7.4.2	J and the second				
		7.4.3	Consistency with data sources				
	7.5	Valida	ation				
		7.5.1					
		7.5.2					
		7.5.3	1				
	7.6		mentation				
			Documentation of data sources				
		7.6.2	Documentation of averaging techniques				
		7.6.3	\mathcal{C}				
			Documentation of benchmark testing				
		7.6.5	Documentation of expected range of qualified application	13			
8	Sum	mary		13			
9	Refe	References					
-	11010						
۸ -	ppen	div					
A			Widely used nuclear data sets	15			
	ADD	SHULX	WIGHT USEG HUCIEAL GATA SETS	1 . 7			