# **American Nuclear Society**

## WITHDRAWN

July 26, 2002 ANSI/ANS-2.8-1992

# determining design basis flooding at power reactor sites

# an American National Standard

No longer being maintained as an American National Standard. This standard may contain outdated material or may have been superseded by another standard. Please contact the ANS Standards Administrator for details.



published by the American Nuclear Society 555 North Kensington Avenue La Grange Park, Illinois 60525 USA

American National Standard for Determining Design Basis Flooding at Power Reactor Sites

Secretariat American Nuclear Society

Prepared by the American Nuclear Society Standards Committee Working Group ANS-2.8

Published by the American Nuclear Society 555 North Kensington Avenue La Grange Park, Illinois 60525 USA

Approved July 28, 1992 by the American National Standards Institute, Inc.

### National Standard

American Designation of this document as an American National Standard attests that the principles of openness and due process have been followed in the approval procedure and that a consensus of those directly and materially affected by the standard has been achieved.

> This standard was developed under the procedures of the Standards Committee of the American Nuclear Society; these procedures are accredited by the American National Standards Institute, Inc., as meeting the criteria for American National Standards. The consensus committee that approved the standard was balanced to ensure that competent, concerned, and varied interests have had an opportunity to participate.

> An American National Standard is intended to aid industry, consumers, governmental agencies, and general interest groups. Its use is entirely voluntary. The existence of an American National Standard, in and of itself, does not preclude anyone from manufacturing, marketing, purchasing, or using products, processes, or procedures not conforming to the standard.

> By publication of this standard, the American Nuclear Society does not insure anyone utilizing the standard against liability allegedly arising from or after its use. The content of this standard reflects acceptable practice at the time of its approval and publication. Changes, if any, occurring through developments in the state of the art, may be considered at the time that the standard is subjected to periodic review. It may be reaffirmed, revised, or withdrawn at any time in accordance with established procedures. Users of this standard are cautioned to determine the validity of copies in their possession and to establish that they are of the latest issue.

> The American Nuclear Society accepts no responsibility for interpretations of this standard made by any individual or by any ad hoc group of individuals. Requests for interpretation should be sent to the Standards Department at Society Headquarters. Action will be taken to provide appropriate response in accordance with established procedures that ensure consensus on the interpretation.

> Comments on this standard are encouraged and should be sent to Society Headquarters.

#### Published by

#### **American Nuclear Society** 555 North Kensington Avenue, La Grange Park, Illinois 60525 USA

Copyright © 1992 by American Nuclear Society.

Any part of this standard may be quoted. Credit lines should read "Extracted from American National Standard ANSI/ANS-2.8-1992 with permission of the publisher, the American Nuclear Society." Reproduction prohibited under copyright convention unless written permission is granted by the American Nuclear Society.

Printed in the United States of America

#### **Foreword** (This Foreword is not a part of the American National Standard for Determining Design Basis Flooding at Power Reactor Sites, ANSI/ANS-2.8-1992.)

The purpose of this document is to specify criteria for determining design basis flooding at power reactor sites. This standard was prepared by Working Group ANS-2.8 of ANS-2 Subcommittee, Site Evaluation, of the American Nuclear Society Standards Committee. The directive to the working group was as follows: "Guidelines are to be developed to establish design basis flooding at power reactor sites as a result of river, stream, or seismically induced dam failure; surge, seiche, or wave action flooding, or any combination of these events. Methodology will be described for evaluating the worst site-related flood at a power reactor site caused by either a probable maximum flood on streams and rivers and any dam failures resulting therefrom: a seismically induced dam failure flood; a probable maximum surge and seiche flood; and any attendant wind-generated wave activity associated with these events, or caused by a reasonable combination of less severe events."

This standard covers material that meets the requirements of Section 2.4, Hydrologic Engineering, of Regulatory Guide 1.70, Revision 3, November 1978, "Standard Format and Content of Safety Analysis Reports for Nuclear Power Plants," issued by the Regulatory Staff of the U.S. Nuclear Regulatory Commission (NRC). This standard does not cover requirements of this Regulatory Guide on the following Standard Format 2.4 sections:

(1) Low Water Considerations—Addressed by American National Standard Evaluation of Surface-Water Supplies for Nuclear Power Sites, ANSI/ANS-2.13-1979 (R1988).

(2) Dispersion, Dilution, and Travel Times of Accidental Releases of Liquid Effluents in Surface Waters—Addressed by American National Standard Evaluation of Radionuclide Transport in Ground Water for Nuclear Power Sites, ANSI/ANS-2.17-1980 (R1989).

(3) Groundwater—Addressed by American National Standard Evaluation of Ground Water Supply for Nuclear Power Sites, ANSI/ANS-2.9-1980 (R1989).

(4) Technical Specifications and Emergency Operation Requirements.

(5) Probable Maximum Tsunami Flooding.

Before preparing the Safety Analysis Report (SAR) Section 2.4, Hydrologic Engineering, for the licensing of nuclear power plants, the applicant should be aware of hydrologic work that has been done by others in the area of interest. Almost invariably, much work can be saved by utilizing all or parts of studies by local, state, and federal agencies. Such information as dimensioned or dimensionless unit hydrographs, loss rates, lag times, historical floods, and geologic and groundwater data, etc., may be obtained from such sources. Sometimes the probable maximum flood has already been derived at the site or at a point near enough to be transposed.

The prime source of such information is the U.S. Army (Corps of Engineers). Other federal agencies that may have useful data are the Bureau of Reclamation, Soil Conservation Service, Weather Service, Geological Survey, Tennessee Valley Authority, Environmental Protection Agency, Federal Energy Regulatory Commission (formerly Federal Power Commission), Federal Emergency Management Agency, and the NRC. Most states have one or more agencies that are concerned with various aspects of water resources. Power companies, particularly those with hydropower capacity, are another source, as are municipal or regional water-supply organizations.

Safety Analysis Reports for other nuclear plants in the area may also provide useful information. It is also profitable to discuss the specific site in detail with the hydrology staff of the NRC prior to starting preparation of Section 2.4.

The first issue of the standard was approved by the American National Standards Institute, Inc., on November 1, 1976, and was published by the American Nuclear Society as American National Standard for Determining Design Basis Flooding at Power Reactor Sites, N170-1976 (ANS 2.8).

The first revision of the standard was approved on February 17, 1981; it was published as American National Standard ANSI/ANS-2.8-1981.

This revision of the standard was developed by a reconstituted working group of ANS-2.8, which had the following members:

R. M. Noble, Chairman, Noble Consultants, Inc. B. J. Buehler, Consultant (formerly Tennessee

Valley Authority)

C. B. Cecilio, Pacific Gas & Electric Company G. B. Dougherty, Consultant

J. P. Jacobson, Yankee Atomic Electric Company Y. J. Tsai, Stone & Webster Engineering Corporation R. Wescott, U.S. Nuclear Regulatory Commission Fei-Fan Yeh, Ebasco Services, Inc.

Subcommittee ANS-2, Site Evaluation, of the American Nuclear Society Standards Committee had the following members at the time it approved this revision:

- J. K. McCall, Chairman, Pacific Gas & Electric Company
- L. L. Beratan, U.S. Nuclear Regulatory Commission A. Brearley, Sargent & Lundy
- L. E. Escalante, Los Angeles Department of Water & Power

M. I. Goldman, NUS Corporation

W. W. Hays, U.S. Geological Survey

- C. R. McClure, Bechtel Civil and Minerals, Inc.
- K. T. McLoughlin, New York Power Pool S. J. Milioti, Impell Corporation

- G. W. Nicholas, Dames and Moore
- R. M. Noble, Noble Consultants, Inc.
- D. Ostrom, Southern California Edison Company
- D. L. Siefken, Roy F. Weston, Inc.
- I. Spickler, U.S. Nuclear Regulatory Commission
- J. D. Stevenson, Stevenson & Associates
- A. Vaish, PMB Systems Engineers, Inc.
- R. W. Whalin, U.S. Army Corps of Engineers
- K. Wiedner, Bechtel Power Corporation

The American Nuclear Society's Nuclear Power Plant Standards Committee had the following membership at the time it of its approval of this standard:

W. T. Ullrich, Chairman M. D. Weber, Secretary

#### Representative

#### Organization

W. M. Andrews	Southern Company Services, Inc.
F. Boorboor	
C. O. Coffer	
L. J. Cooper	
J. D. Crawford	Combusion Engineering
W. H. D'Ardenne (Vice-chairman)	General Electric Company
S. N. Ehrenpreis	
S. B. Gerges	NUS Corporation
C. J. Gill	Bechtel National, Inc.
C. E. Johnson	
(Vacant)	
R. T. Lancet	Rockwell International Corporation
J. C. McCall	Pacific Gas & Electric Company
J. F. Mallay Adv	anced Technology Engineering Systems, Inc.
R. E. Miller	Duke Power Company
J. A. Nevshemal	
P. T. Reichert	Stearns Catalytic, Inc.
B. M. Rice	
(for the Institut	e of Electrical & Electronics Engineers, Inc.)
J. C. Saldarini	Ebasco Services, Inc.
M. O. Sanford	GPU Nuclear Corporation
S. L. Stamm	
J. D. Stevenson	Stevenson & Associates
(	for the American Society of Civil Engineers)
T. J. Sullivan	Institute of Nuclear Power Operations
C. D. Thomas, Jr	Yankee Atomic Electric Company
W. T. Ullrich	Philadelphia Electric Company
	(for the American Nuclear Society)
G. P. Wagner	Commonwealth Edison Company
G. L. Wessman	Torrey Pines Technology
G. J. Wrobel	Rochester Gas & Electric Corporation

Contents	Section Pa 1. Introduction and Scope	age
	1. Introduction and Scope         1.1 Scope         1.2 Discussion	1
	2. Definitions	1
	<ol> <li>Hydrologic Description</li></ol>	2
	<ul> <li>4. Plant Safety from Floods</li></ul>	3 4 5 5
	<ol> <li>5. Probable Maximum Flood from Precipitation</li> <li>5.1 General</li> <li>5.2 Probable Maximum Precipitation</li> <li>5.3 Snowpack and Snowmelt Flood</li> <li>5.4 Runoff and Stream-Course Models</li> <li>5.5 Hydrologic Dam Failures</li> <li>5.6 Sediment Transport</li> <li>5.7 Coincident Wind-Wave Activity</li> <li>5.8 Probable Maximum Flood Summary</li> </ol>	5 6 7 .10 .11 .13 .13
	<ul> <li>6. Nonhydrologic Dam Failures</li> <li>6.1 General</li> <li>6.2 Seismic Dam Failures</li> <li>6.3 Dam Failures from Other Causes</li> <li>6.4 Failure and Failure Surge Analysis</li> <li>6.5 Summary</li> </ul>	.14 .15 .16 .16
	<ul> <li>7. Probable Maximum Surge and Seiche Flooding</li></ul>	.17 .17 .19 .23
	8. Ice Flooding         8.1 General         8.2 Ice Effect         8.3 Surface Ice	. 29 . 29
	<ul> <li>9. Combined Events Criteria</li></ul>	. 29

10. Cooling Water Canals and Reservoirs	
10.1 General	
10.2 Canals	
10.3 Reservoirs	
an na analas an	
11. Plant Site Drainage	
11.1 General	
11.2 Factors for Consideration	
11.3 Site Drainage Area	
11.4 Roof Drainage	
11.5 Grades and Drains	
11.6 Local Intense Precipitation	
11.7 Model	
11.8 Snow and Ice Accumulation	
11.9 Summary	
12. References	
Appendix A—Bibliography	
Appendix B-Interpreted Probabilities of Combined Events	
Tables	
Table 1   Example for 3.1.3	
Table 3    Hawaii Fastest Mile Quantiles    31	
Table 4    Puerto Rico Fastest Mile Quantiles    31	
Figure 1 Isotach 0.50 Quantiles (in miles per hour): Annual Extreme-	
Mile, 30 ft Above Ground, 2-yr Mean Recurrence Interval	

### Determining Design Basis Flooding at Power Reactor Sites

#### 1. Introduction and Scope

1.1 Scope. This document presents criteria to establish design basis flooding for nuclear safetyrelated features at power reactor sites. Methodology is described to evaluate the flood having virtually no risk of exceedance that can be caused by precipitation and snowmelt and any resulting dam failures; seismically induced dam failures; surge or seiche and attendant wind-generated wave activity; or a reasonable combination of these events.

**1.2 Discussion.** This standard covers that material necessary to develop the design basis flooding for use in the evaluation of the adequacy of a nuclear power plant site. Water-related effects such as water levels, waves, wave forces, ice, erosion, and sedimentation are included to assist in the design of safety-related facilities. Where information presentation requirements are stated in this standard, such as "provide," "tabulate," or "describe," such information shall be provided as a part of the documentation for the design basis flood estimate.

1.2.1 Exclusions. This standard does not cover:

(1) Probable maximum tsunami flooding.

(2) Low water considerations.

(3) Dispersion, dilution, and travel times of accidental releases of liquid effluents.

(4) Groundwater.

(5) Technical specification and emergency operation requirements.

(6) Channel diversions.

(7) Flooding protection requirements (partially addressed).

(8) Flooding from pipe rupture or on-site tank failures.

**1.2.2 Probabilistic Approach.** In this standard, guidelines to determine design basis floods are primarily associated with "probable maximum" events of deterministic origins. The standard does not include guidelines for using a probabilistic approach, including stochastic techniques. It does, however, recommend in 9.1.2 a target annual exceedance probability of less than  $1 \times 10^{-6}$  for selecting combined events that collectively comprise design bases floods.

At the time this standard was prepared, there were no recognized procedures to accurately and objectively define the exceedance probabilities of significant rare events included in probable maximum analysis. As data and procedures improve, probabilistic approaches are encouraged. The exclusion of probabilistic approaches from this standard should not be construed in a manner to inhibit innovation because preferred methodology for a particular case could likely be beneficial and acceptable for a specific site.

#### 2. Definitions

**moving squall line.** A line or narrow band of active thunderstorms having a pressure jump with the cold front providing the initial piston-like impetus, and a mature instability line that is located in the warm sector of a wave cyclone about 50 to 200 miles in advance of a cold front usually oriented roughly parallel to the cold front and moving in about the same direction and speed as the cold front.

**probable maximum flood (PMF).** The hypothetical flood (peak discharge, volume, and hydrograph shape) that is considered to be the most severe reasonably possible, based on comprehensive hydrometeorological application of probable maximum precipitation and other hydrologic factors favorable for maximum flood runoff such as sequential storms and snowmelt.

**probable maximum gradient wind.** A probable gradient wind of a designated duration above the surface friction layer, of which there is virtually no risk of being exceeded. The event may be considered to have a probability of occurence comparable to that of a probable maximum precipitation.

**probable maximum hurricane (PMH).** A hypothetical hurricane having that combination of characteristics that makes it the most severe that can reasonably occur in the particular region involved. The hurricane approaches the point under study along a critical path and at an optimum rate of movement, which results in the most adverse flooding.