

ERRATUM ISSUED

Find erratum inside front cover. If erratum is missing, contact the ANS Standards Department at Standards@ans.org or 708-579-8269 for replacement copy.

American Nuclear Society

WITHDRAWN

February 27, 2016
ANSI/ANS-57.5-1996 (R2006)

**light water reactors fuel assembly
 mechanical design and evaluation**

an American National Standard

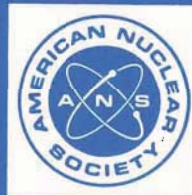
This standard has been reviewed and reaffirmed with the recognition that it may reference other standards and documents that may have been superseded or withdrawn. The requirements of this document will be met by using the version of the standards and documents referenced herein. It is the responsibility of the user to review each of the references and to determine whether the use of the original references or more recent versions is appropriate for the facility. Variations from the standards and documents referenced in this standard should be evaluated and documented.

This standard does not necessarily reflect recent industry initiatives for risk informed decision-making or a graded approach to quality assurance. Users should consider the use of these industry initiatives in the application of this standard.

REAFFIRMED

February 28, 2006
ANSI/ANS-57.5-1996; R2006

No longer being maintained as an American National Standard. This standard may contain outdated material or may have been superseded by another standard. Please contact the ANS Standards Administrator for details.



published by the
 American Nuclear Society
 555 North Kensington Avenue
 La Grange Park, Illinois 60525 USA

ERRATUM

ANSI/ANS-57.5-1996 (R2006)

Light Water Reactors Fuel Assembly Mechanical Design and Evaluation

An error was identified in Section 5.4.1.3 on page 6. The preceding explanatory text is not consistent with the equation. Section 5.4.1.3 should read as follows:

5.4.1.3 For time-dependent effects, the summation of actual time (t) at a given stress level divided by the time of failure (t_f) at that stress level shall be less than 1.0. Also, the summation of creep strain incurred (ϵ^c) divided by the creep strain to failure

(ϵ_f^c) shall be less than 1.0.

$$\sum \frac{t_i}{(t_f)_i} < 1.0 \quad \text{and} \quad \sum \frac{\epsilon_i^c}{(\epsilon_f^c)_i} < 1.0$$

**American National Standard for
Light Water Reactors Fuel Assembly
Mechanical Design and Evaluation**

Secretariat
American Nuclear Society

Prepared by the
**American Nuclear Society
Standards Committee
Working Group ANS-57.5**

Published by the
**American Nuclear Society
555 North Kensington Avenue
La Grange Park, Illinois 60526 USA**

Approved February 8, 1996
by the
American National Standards Institute, Inc.

American National Standard

Designation of this document as an American National Standard attests that the principles of openness and due process have been followed in the approval procedure and that a consensus of those directly and materially affected by the standard has been achieved.

This standard was developed under the procedures of the Standards Committee of the American Nuclear Society; these procedures are accredited by the American National Standards Institute, Inc., as meeting the criteria for American National Standards. The consensus committee that approved the standard was balanced to ensure that competent, concerned, and varied interests have had an opportunity to participate.

An American National Standard is intended to aid industry, consumers, governmental agencies, and general interest groups. Its use is entirely voluntary. The existence of an American National Standard, in and of itself, does not preclude anyone from manufacturing, marketing, purchasing, or using products, processes, or procedures not conforming to the standard.

By publication of this standard, the American Nuclear Society does not insure anyone utilizing the standard against liability allegedly arising from or after its use. The content of this standard reflects acceptable practice at the time of its approval and publication. Changes, if any, occurring through developments in the state of the art, may be considered at the time that the standard is subjected to periodic review. It may be reaffirmed, revised, or withdrawn at any time in accordance with established procedures. Users of this standard are cautioned to determine the validity of copies in their possession and to establish that they are of the latest issue.

The American Nuclear Society accepts no responsibility for interpretations of this standard made by any individual or by any ad hoc group of individuals. Requests for interpretation should be sent to the Standards Department at Society Headquarters. Action will be taken to provide appropriate response in accordance with established procedures that ensure consensus on the interpretation.

Comments on this standard are encouraged and should be sent to Society Headquarters.

Published by

American Nuclear Society
555 North Kensington Avenue, La Grange Park, Illinois 60526 USA

Copyright © 1996 by American Nuclear Society.

Any part of this standard may be quoted. Credit lines should read "Extracted from American National Standard ANSI/ANS-57.5-1996 with permission of the publisher, the American Nuclear Society." Reproduction prohibited under copyright convention unless written permission is granted by the American Nuclear Society.

Printed in the United States of America

Foreword

(This Foreword is not a part of American National Standard for Light Water Reactors Fuel Assembly Mechanical Design and Evaluation, ANSI/ANS-57.5-1996.)

This American National Standard provides a procedure for determining the mechanical adequacy of fuel assembly designs for light water nuclear reactors. Specific requirements for design and specific rules for demonstrating compliance are also included.

It is not the intent of this standard to endorse any design feature, material, material property information, analysis method, or other procedure, or in any way to inhibit development or innovation in any of these areas. However, this standard does include certain requirements intended to ensure that the methods or material properties which are used are appropriate and adequately documented.

Suggestions for improvement of this standard are welcome. They should be sent to the American Nuclear Society, 555 North Kensington Avenue, La Grange Park, Illinois 60526.

The membership of Working Group ANS-57.5, at the time it submitted this revision of this standard, was as follows:

R. H. Ripley, Chairman, *Union Electric Company*
J. A. Nevshemal, *Raytheon UE&C*

The American Nuclear Society's Nuclear Power Plant Standards Committee (NUPPSCO) had the following membership at the time of its approval of the standard:

W.H. D'Ardenne, Chairman
M.D. Weber, Secretary

R.E. Allen	UE&C Nuclear (for the Institute of Electrical and Electronics Engineers, Inc.)
P.L. Ballinger	Nebraska Public Power District
F. Boorboor	Nuclear Placement Services, Inc.
J.C. Bradford	Bechtel
R.H. Bryan, Jr.	Tennessee Valley Authority
T.W.T. Burnett	Westinghouse Electric Corporation
J.D. Cohen	Westinghouse Savannah River Company
J.B. Cotton	Philadelphia Electric Company
W.H. D'Ardenne	DAE Enterprises (for the American Nuclear Society)
L.E. Davis	Commonwealth Edison Company
M. Drouin	U.S. Nuclear Regulatory Commission
P.H. Hepner	ABB/Combustion Engineering, Inc.
R.A. Hill	G.E. Nuclear Energy
J.T. Luke	Florida Power & Light Company
J.F. Mallay	Liberty Consulting Group
C.H. Moseley, Jr.	Performance Development Corporation
J.A. Nevshemal	Raytheon UE&C
W.N. Prillaman	Babcock & Wilcox Company
W.C. Ramsey, Jr.	Southern Company Services, Inc.
W.B. Reuland	Mollerus Engineering Corporation
R.F. Sacramo	Halliburton NUS Corporation
J.C. Saldarini	Raytheon Engineers & Constructors
J. Savy	Lawrence Livermore National Laboratory
R.E. Scott	Scott Enterprises
D.J. Spellman	Oak Ridge National Laboratory
S.L. Stamm	Stone & Webster Engineering Corporation
J.D. Stevenson	Stevenson & Associates
C.D. Thomas, Jr.	Yankee Atomic Electric Company
G.P. Wagner	Commonwealth Edison Company
N. Weber	Consultant
G.J. Wrobel	Rochester Gas & Electric Corporation

Contents	Section	Page
	1. Scope	1
	2. Purpose	1
	3. Definitions	1
	4. Compliance	1
	5. Design and Evaluation	2
	5.1 Design Conditions	2
	5.2 Functional Requirements	2
	5.3 Design Parameters	3
	5.4 Limits and Margins	6
	5.5 Specific Requirements for Design	7
	6. Documentation Requirements	11
	6.1 Objectives	11
	6.2 Content	11
	7. References	12
	Appendices	
	Appendix A Design Condition Events	13
	Appendix B Illustration of the Use of the Standard	17
	Tables	
	Table 1 Matrix	18

Light Water Reactors Fuel Assembly Mechanical Design and Evaluation

1. Scope

This standard sets forth a series of design conditions and functional requirements for the design of fuel assemblies for light water cooled commercial power reactors. It includes specific requirements for design, as well as design criteria to ensure adequate fuel assembly performance. The standard establishes a procedure for performing an evaluation of the mechanical design of fuel assemblies. It does not address the various aspects of neutronic or thermal-hydraulic performance except where these factors impose loads or constraints on the mechanical design of the fuel assemblies.

2. Purpose

The purpose of this standard is to establish a set of design requirements for the mechanical design of initial core or reload fuel assemblies and for the evaluation of that design.

These design requirements include:

- (1) A comprehensive set of functional requirements for fuel assemblies
- (2) A method for selecting the specific events in each of the design conditions
- (3) A comprehensive list of parameters, including material properties, chemical reactions, irradiation effects, and failure modes, which could affect the capability of fuel assemblies to satisfy one or more functional design requirements
- (4) A method to achieve two goals: define which parameters and assumptions affect the capability of the fuel assembly to fulfill each functional requirement under each postulated event, and establish an appropriate limit for the defined parameters and assumptions which ensures that some aspect of a functional requirement for that event is met
- (5) A procedure to document that the fuel assembly design has been evaluated in accordance with the design limits discussed in 5.4

(or similar) and has been shown to fulfill each functional requirement for each event.

3. Definitions

designer. The organization that has the responsibility for preparing the fuel assembly design.

design parameters. Material properties, dimensional characterizations, or physical response phenomena necessary to describe or evaluate fuel assembly behavior.

event. A describable situation that must be accounted for in design.

fuel assembly. The smallest modular unit comprised of individual fuel rods and associated integral component parts for handling, control, support, and maintenance of the unit's geometry. For boiling water reactors, the channel that encloses the fuel bundle and the channel fastener is included as part of the fuel assembly for design purposes.

functional requirement. One of several required capabilities of a fuel assembly that is necessary to meet its design function.

limit. A bounding value of a variable or parameter, which is established to ensure that one or more aspects of a functional requirement are satisfied.

margin. A quantitative relationship between a design evaluation result for a given event and a limit associated with a functional requirement.

shall, should, and may. The word "shall" denotes a requirement; the word "should" denotes a recommendation; and the word "may" denotes permission, neither a requirement nor a recommendation.

4. Compliance

Design documentation shall be prepared to show how the criteria and requirements of this standard are satisfied. Provisions for dissemination of