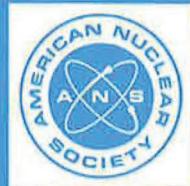


American National Standard

guidelines on the nuclear analysis
and design of concrete radiation shielding
for nuclear power plants

WITHDRAWN

No longer being maintained as an American National Standard. This standard may contain outdated material or may have been superseded by another standard. Please contact the ANS Standards Administrator for details.



published by the
American Nuclear Society
555 North Kensington Avenue
La Grange Park, Illinois 60525 USA

**ANSI/ANS-6.4-1977
(N403)**

**American National Standard
Guidelines on the Nuclear Analysis
and Design of Concrete Radiation Shielding
for Nuclear Power Plants**

**Secretariat
American Nuclear Society**

**Prepared by the
American Nuclear Society
Standards Committee
Working Group ANS-6.4**

**Published by the
American Nuclear Society
555 North Kensington Avenue
La Grange Park, Illinois 60525 USA**

**Approved August 8, 1977
by the
American National Standards Institute, Inc.**

Abstract

Ordinary concrete is by far the most widely used radiation shielding material in commercial nuclear power plants. The purpose of this Standard is to provide guidance and recommendations on concrete shielding analysis and design. It is directed primarily toward shielding designers. The standard describes shielding concretes, summarizes shielding data, discusses calculational methods, and covers applications such as bulk shielding and reflection problems. Where possible, the standard makes specific recommendations; taken as a whole, it constitutes a guide to good practice in concrete shield design.

American National Standard

An American National Standard implies a consensus of those substantially concerned with its scope and provisions. An American National Standard is intended as a guide to aid the manufacturer, the consumer, and the general public. The existence of an American National Standard does not in any respect preclude anyone, whether he has approved the standard or not, from manufacturing, marketing, purchasing, or using products, processes, or procedures not conforming to the standard. American National Standards are subject to periodic review and users are cautioned to obtain the latest editions.

CAUTION NOTICE: This American National Standard may be reviewed or withdrawn at any time. The procedures of the American National Standards Institute require that action be taken to reaffirm, revise, or withdraw this Standard no later than five years from the date of publication. Purchasers of this Standard may receive current information, including interpretation, on all standards published by the American Nuclear Society by calling or writing to the Society.

Published by

**American Nuclear Society
555 North Kensington Avenue, La Grange Park, Illinois 60525 USA**

Price: \$38.00

Copyright © 1978 by American Nuclear Society

Any part of this Standard may be quoted. Credit lines should read "Extracted from American National Standard ANSI/ANS-6.4-1977 with permission of the publisher, the American Nuclear Society." Reproduction prohibited under copyright convention unless written permission is granted by the American Nuclear Society.

Printed in the United States of America

Foreword

(This Foreword is not a part of Guidelines on the Nuclear Analysis and Design of Concrete Radiation Shielding for Nuclear Power Plants, ANSI/ANS-6.4-1977)

The need for this Standard was identified in mid-1972 by D. K. Trubey, Chairman of Subcommittee ANS-6, Radiation Protection and Shielding. The existing standard ANSI N101.6-1972, "Concrete Radiation Shields," provides excellent guidance on the construction of concrete radiation shielding structures, but contains almost no information on shielding effectiveness or analysis. The following standard can be considered supplementary to ANSI N101.6-1972.

This Standard is meant to be a "guide to good practice" in the area of concrete shielding analysis and design. Recommendations are given where possible, but more often the choice of analytical methods must be left to the discretion of the shielding engineer as appropriate to the particular job, whether it be a conceptual design or final construction drawing.

This Standard was compiled and written by the Working Group ANS-6.4 of the American Nuclear Society, which had the following members at the time it prepared and approved this Standard:

B. A. Engholm, *Chairman, General Atomic*
W. L. Bunch, *Westinghouse, Hanford*
A. I. Cygelman, *Burns and Roe*
W. R. Johnson, *University of Virginia*

M. J. Kolar, *Commonwealth Associates*
E. Normand, *Sargent and Lundy*
E. A. Warman/E. T. Boulette, *Stone & Webster*
N. B. Willoughby, *Bechtel, Gaithersburg*

The membership of Subcommittee ANS-6 at the time of their approval of this Standard was:

D. K. Trubey, *Chairman, Oak Ridge National Laboratory*
M. E. Battat, *Los Alamos Scientific Laboratory*
George G. Biro, *Gills and Hill, Inc.*
B. A. Engholm, *General Atomic Company*

H. E. Hungerford, *Purdue University*
P. J. Persiani, *Argonne National Laboratory*
D. J. Schuh, II, *Nuclear Measurements Corporation*
G. L. Siminons, *Science Applications, Inc.*
E. A. Warman, *Stone & Webster Engineering Corporation*

The American National Standards Committee N17, Research Reactors, Reactor Physics and Radiation Shielding, had the following membership when it reviewed and approved this standard:

W. L. Whittemore, *Chairman*
R. S. Carter, *Secretary*

<i>Organization Represented</i>	<i>Name of Representative</i>
American College of Radiology.....	Michel M. TerPogossian
American Institute of Chemical Engineers.....	Richard Duffy
American Nuclear Society	W. L. Whittemore
American Physical Society	W. W. Havens, Jr. Herbert Goldstein (Alt)
American Public Health Association.....	Charles G. Amato William A. Holt (Alt)
American Society of Mechanical Engineers.....	Roy A. Axford
American Society of Radiologic Technologists.....	John H. Tolan
Health Physics Society	Charles A. Willis
Institute of Electrical & Electronics Engineers	H. A. Thomas
National Bureau of Standards	R. S. Carter Tawfik M. Raby (Alt)
National Council on Radiation Protection & Measurement.....	Arthur B. Chilton
U.S. Nuclear Regulatory Commission	Karl R. Goller Robert J. Schemel (Alt)
U.S. Energy Research & Development Administration	Phillip B. Hemmig John W. Lewellen (Alt)
Individual Members.....	J. E. Olhoeft Alfred M. Perry E. A. Warman

Contents	Section	Page
1. Scope	1	
2. Requirements and Recommendations	1	
2.1 Conformance	1	
2.2 Requirements	1	
2.3 Recommendations	2	
3. Standards of Documentation	2	
3.1 Shielding Design Summary Documentation	2	
4. Terms and Definitions	3	
5. Characterization of Concrete	5	
5.1 Introduction	5	
5.2 Concrete Placement	5	
5.3 Water Content	6	
5.4 Heating Effects	6	
5.5 Reinforcing Steel	7	
5.6 Aggregates	7	
6. Calculation Methods	10	
6.1 Introduction	10	
6.2 Point Kernal Methods	10	
6.3 Discrete Ordinates Method	12	
6.4 Monte Carlo Methods	14	
6.5 Other Methods	15	
7. Concrete Shielding Data	18	
7.1 Introduction	18	
7.2 Gamma-Ray Attenuation Coefficients	18	
7.3 Gamma-Ray Buildup Factors	19	
7.4 Secondary Gamma-Ray Production	19	
7.5 Neutron Cross Sections	19	
7.6 Neutron Attenuation Curves	19	
8. Applications	20	
8.1 Radiation Effects	20	
8.2 Minimum Water Content	21	
8.3 Bulk Transport	21	
8.4 Radiation Streaming Through Penetrations	23	
8.5 Reflection	25	
9. References	25	
Bibliography	29	
Appendix A List of Codes	30	
Appendix B Shielding Data	32	
Appendix C Applications Data and Results	51	

Table 5.1 Typical Concrete Properties	9
Figure 6.1 P_1 Convergence of Fast Neutron Dose Rate	16
Figure 6.2 Transport <i>vs.</i> Diffusion Dose Rates in Concrete	17
Table B.1 Mass Attenuation Coefficients for Ordinary Concrete	32
Table B.2 Mass Attenuation Coefficients for Elements Comprising Ordinary Concrete	33
Table B.3 Mass Attenuation Coefficients of Elements Which May Be Found in Other Concrete Compositions	38
Table B.4 Mass Energy-Absorption Coefficients for Ordinary Concrete	40
Table B.5 Gamma Ray Energy Absorption Coefficients of the Concretes	40
Table B.6 The Equivalent Atomic Number for Ordinary Concrete As A Function of Source Energy	41
Table B.7 Buildup Factor Parameters — Dose, 20 mfp	41
Table B.8 Buildup Factor Parameters — Absorption, 20 mfp	42
Table B.9 Buildup Factor Parameters — Dose, 40 mfp	43
Table B.10 Buildup Factor Parameters — Absorption, 40 mfp	43
Table B.11 Gamma-ray Spectra from Thermal Neutron Capture in the Concretes.....	44
Table B.12 Neutron Constants for Concretes.....	45
Figure B.1 Dose Buildup Factor, Point Isotropic Source	46
Figure B.2 Dose Buildup Factor, Plane Isotropic Source	47
Figure B.3 Energy Absorption Buildup Factor, Point Source	48
Figure B.4 Energy Absorption Buildup Factor, Plane Source	49
Figure B.5 Discrete Ordinates <i>vs.</i> Moments Methods Calculation	50
Table C.1 Composition of Hanford Ordinary Concrete As A Function of Temperature	51
Table C.2 Compositions of Concretes Used in Measurements	51
Table C.3 ANISN Spherical Model for PWR Calculation	52
Table C.4 Material Compositions for PWR Calculation	52
Table C.5 Neutron Source Distribution for PWR Calculation.....	53
Table C.6 Neutron Source Spectrum for PWR Calculation	53
Table C.7 Albedo Method Parameters.....	54
Table C.8 Constants for the Expression Fitting the Maerker- Muckenthaler Differential Dose Albedo Data for Fast Neutrons Incident on Concrete	55
Table C.9 Constants for the Expressions Fitting the Coleman et al. Differential and Total Albedo Data for Intermediate-Energy Incident on Reinforced Concrete	56
Table C.10 Values of Parameters for Chilton-Huddleston Gamma-Ray Differential Albedo Formula.....	56
Figure C.1 Resonance Neutron Flux as Function of Temperature	57
Figure C.2 Fast Neutron Flux as Function of Temperature.....	58
Figure C.3 Neutron Fluxes <i>vs.</i> Distance in Concrete (Hanford)	59
Figure C.4 Fluxes <i>vs.</i> Distance in Concrete (USSR)	60
Figure C.5 Radiation Levels in Bradwell Side Shield	61
Figure C.6 Dose Rates in Typical PWR Shield	62
Figure C.7 Gamma Doses from 4-Inch Schedule 160 Pipe	63
Figure C.8 Gamma Doses from 24-Inch Schedule 160 Pipe	64
Figure C.9 Gamma Doses from 6-Foot O.D. Tank	65
Figure C.10 Penetration Types	66
Figure C.11 Geometry For Albedos	67
Figure C.12 Reflection Geometries	68